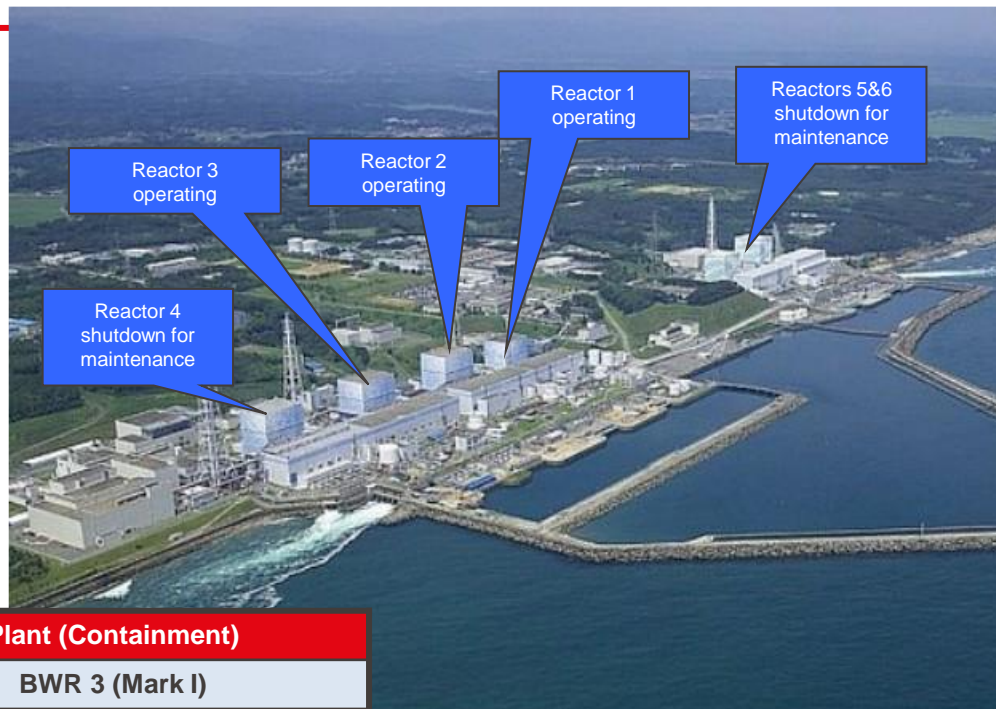


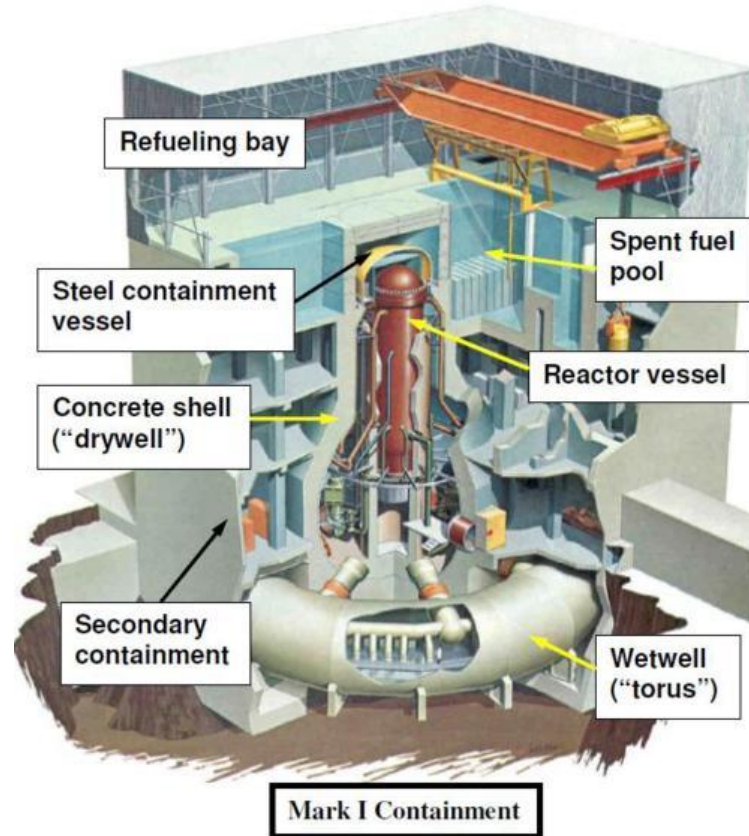


Severe Accidents: Fukushima

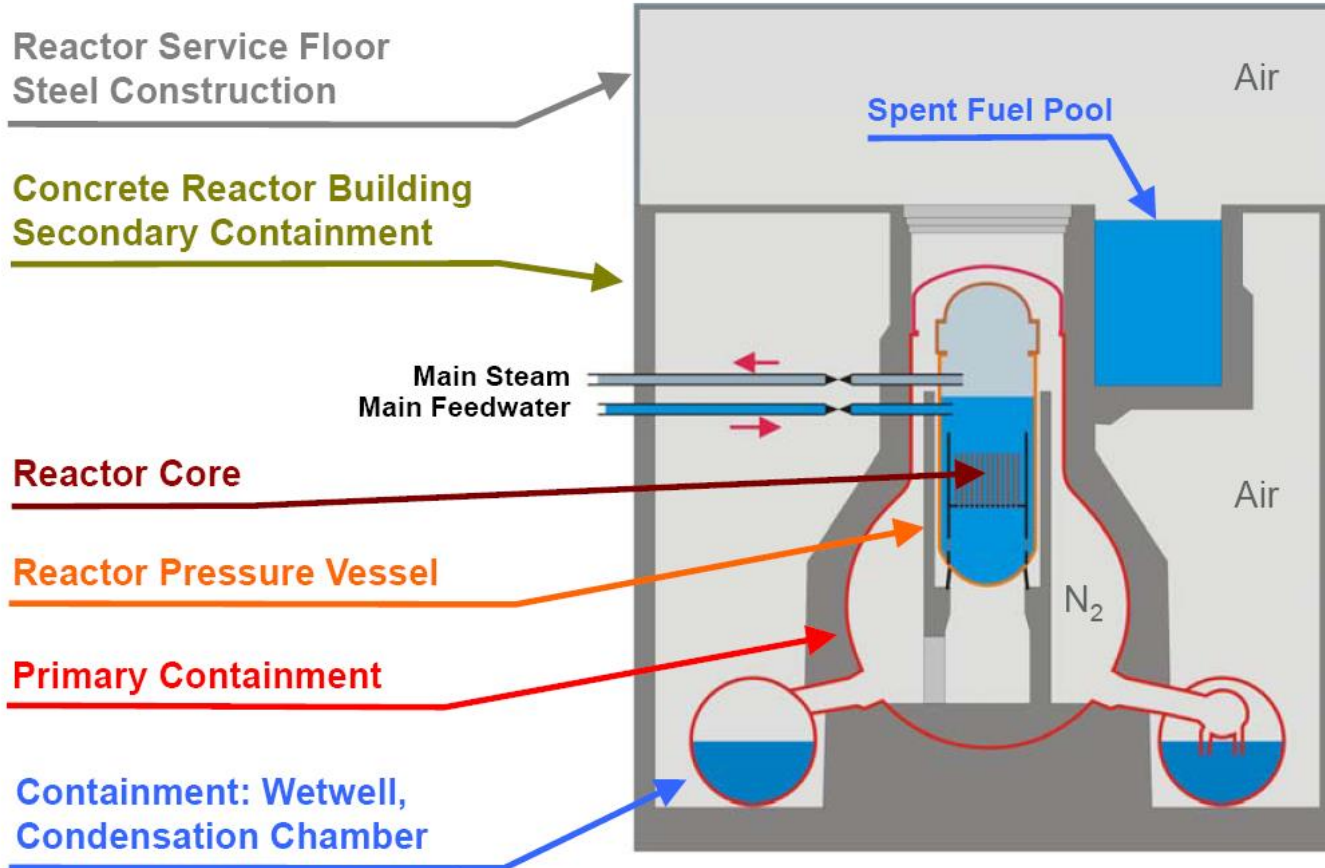
- A scenario with a very low probability ($\sim 10^{-5}$ /reactor-year for internally initiated events) in the domain of Beyond Design Basis Accidents (BDBA)
- Can lead to significant damage to fuel and core materials (partial or complete core melting).
- It may then have serious consequences such as:
 - Loss of containment integrity.
 - Release of radioactive elements into the environment.
- Three major severe accidents:
 1. Three Mile Island (1979)
 2. Chernobyl (1986)
 3. **Fukushima Daiichi (2011)**



Bloc	Commissioned	Power MWe	Hersteller	Plant (Containment)
1	1971	460	GE	BWR 3 (Mark I)
2	1974	784	Toshiba/GE	BWR 4 (improved Mark I)
3	1976	784	Toshiba	BWR 4 (improved Mark I)
4	1978	784	Hitachi	BWR 4 (improved Mark I)
5	1978	784	Toshiba	BWR 4 (improved Mark I)
6	1979	1100	Toshiba/GE	BWR 5 (Mark II)



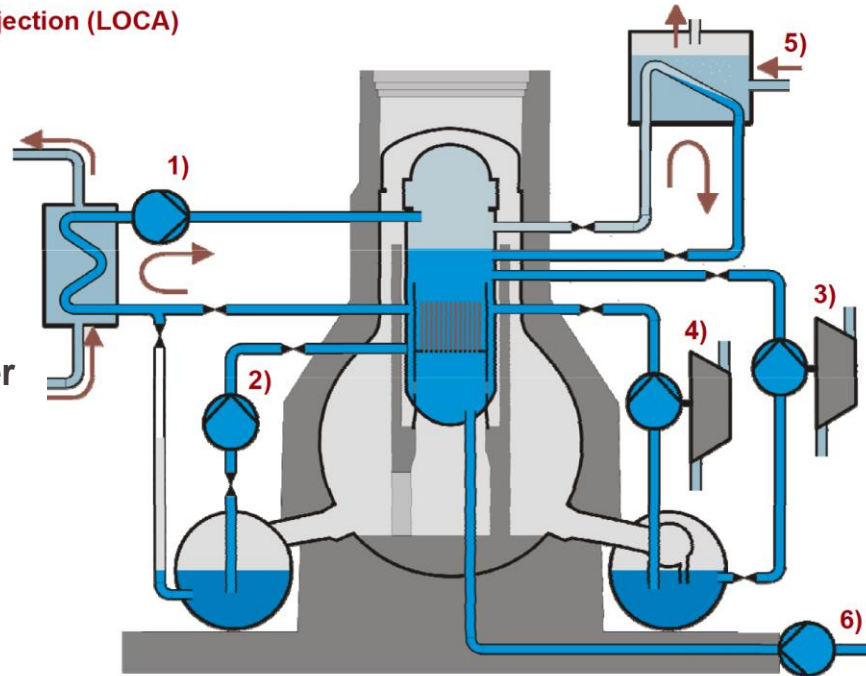
Mark I Containment



- 1) Residual Heat Removal System
- 2) Low-Pressure Core Spray (LOCA)
- 3) High-Pressure Coolant Injection (LOCA)
- 4) Reactor Core Isolation Cooling (Unit 2/3: BWR-4)
- 5) Isolation Condenser (Unit 1: BWR-3)
- 6) Borating System

Electrical power
necessary

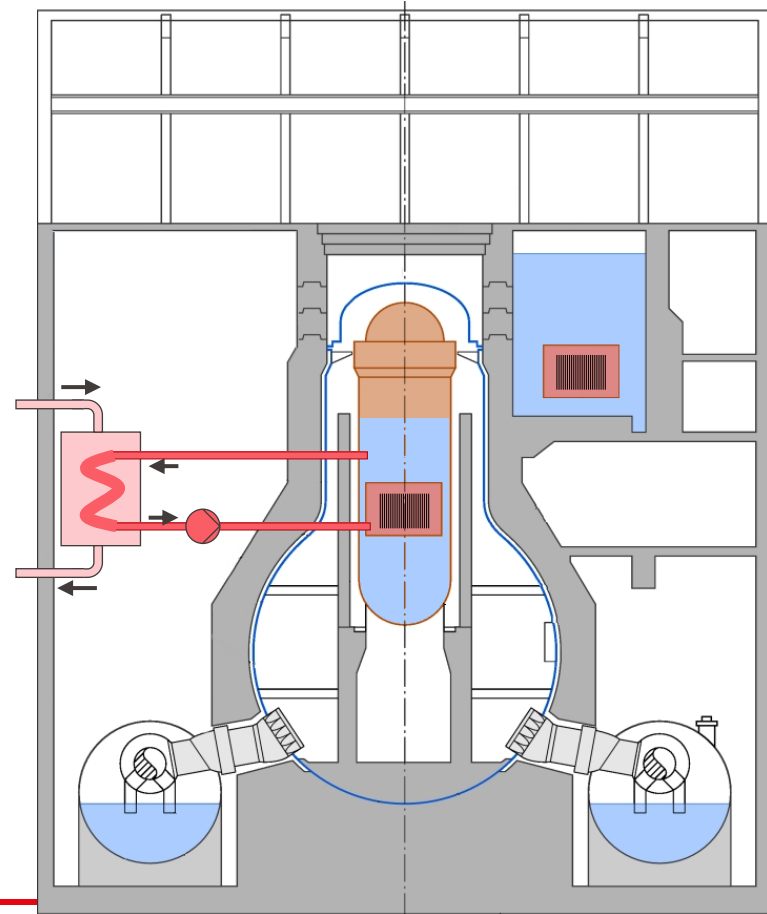
Electrical power
necessary

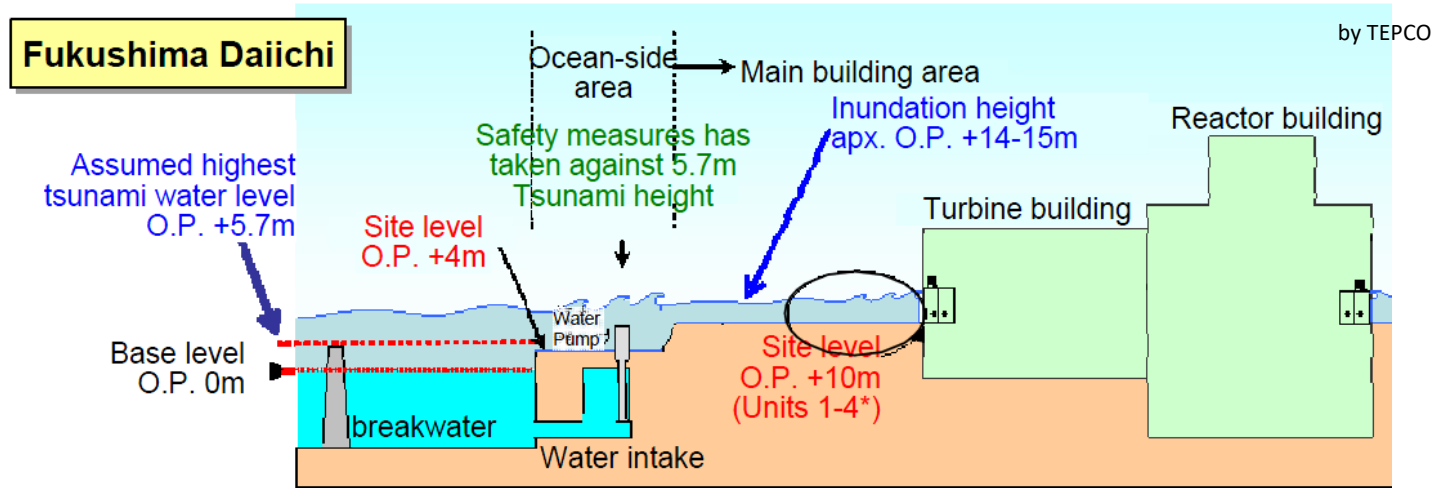


- Epicenter 163 km North-East of Fukushima Daiichi NPP.
- Magnitude 9 (Richter Scale) – Severe destructions in the country but all reactor units are mainly undamaged.
- Power grid fails and reactors are safely scrammed (Daiichi Blocks 4-6 were already in shutdown).

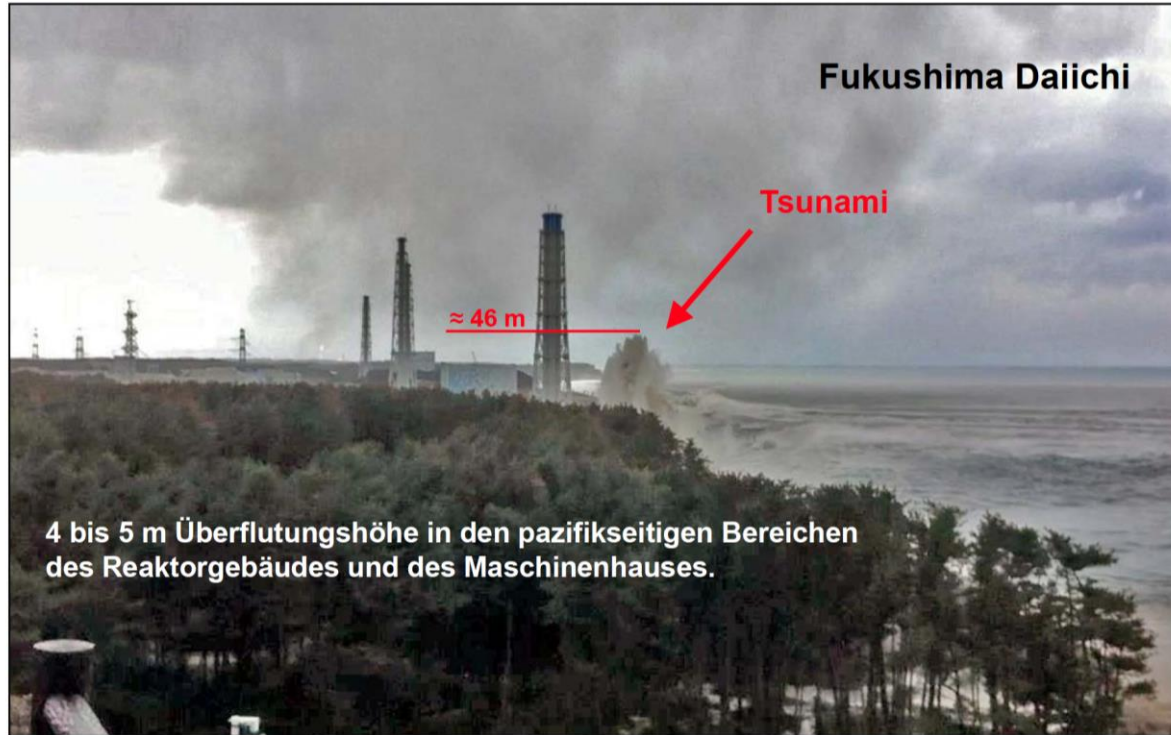


- Reactors are shutdown but decay heat continues to generate power (~1% after 1 day)
- Blackout – loss of external power supply
 - Containment is isolated.
 - Diesel generators are started and drive the ECCS.
- At this point in time, no severe loss of safety functions and plant is in a stable state

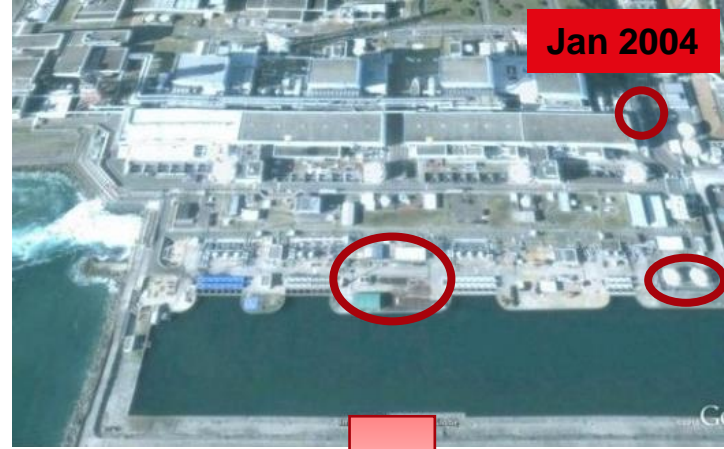


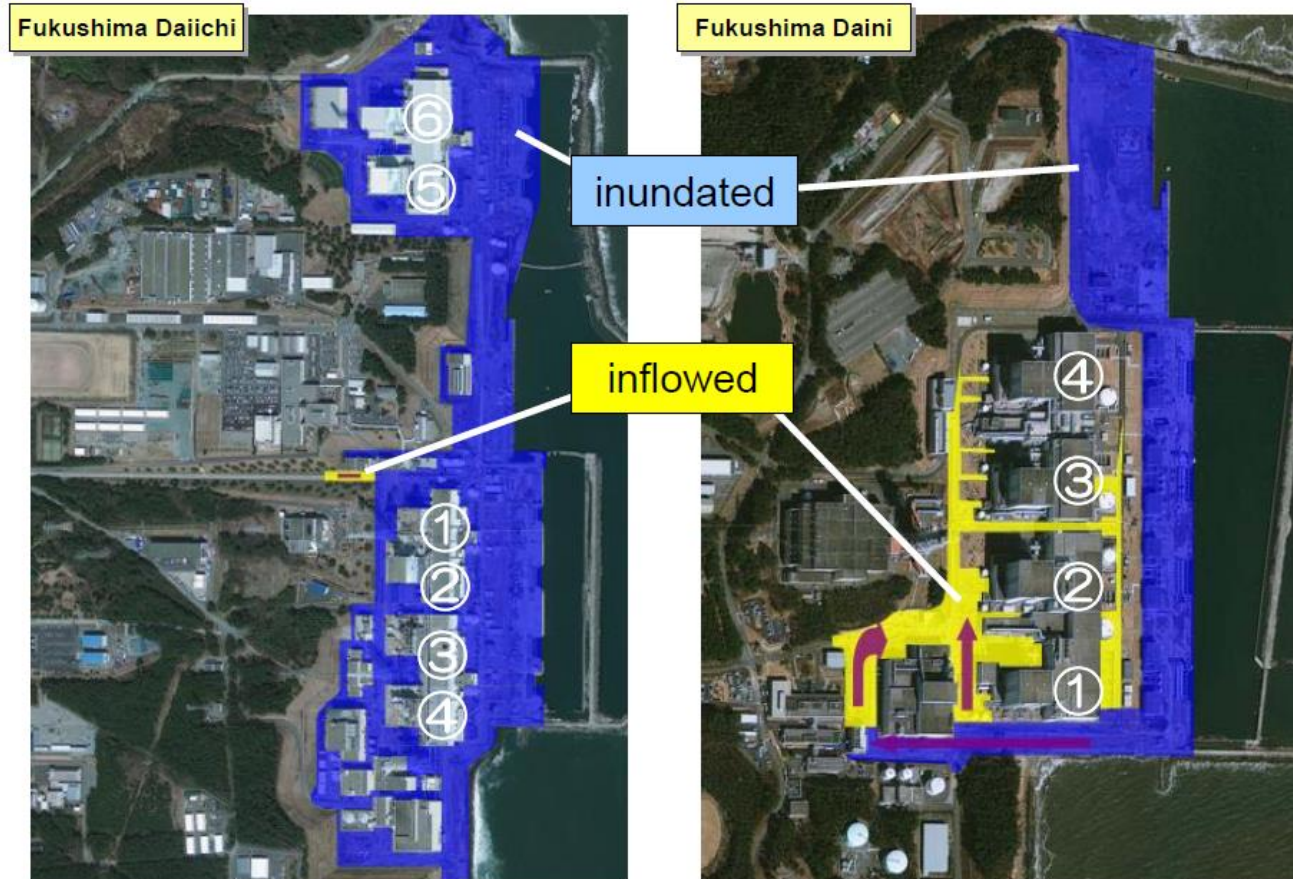


- Height of flood wave 14 m against a site level of 10m...
- Loss of all emergency diesel generators (inundated) → loss of all cooling functions.
- Area non-accessible due to debris everywhere and no access from the outside.

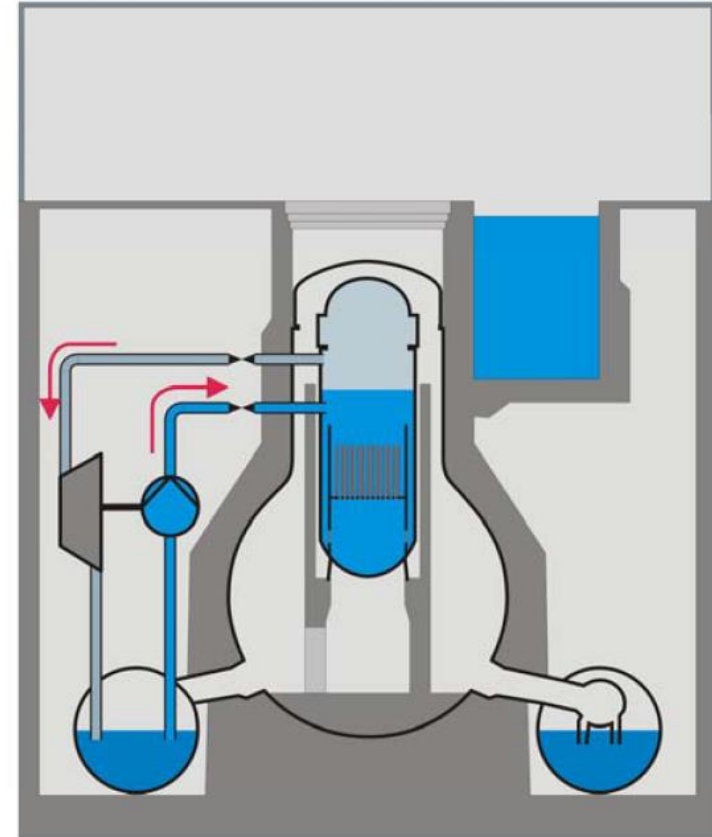


The Tsunami on March 11, 2011, 15:41



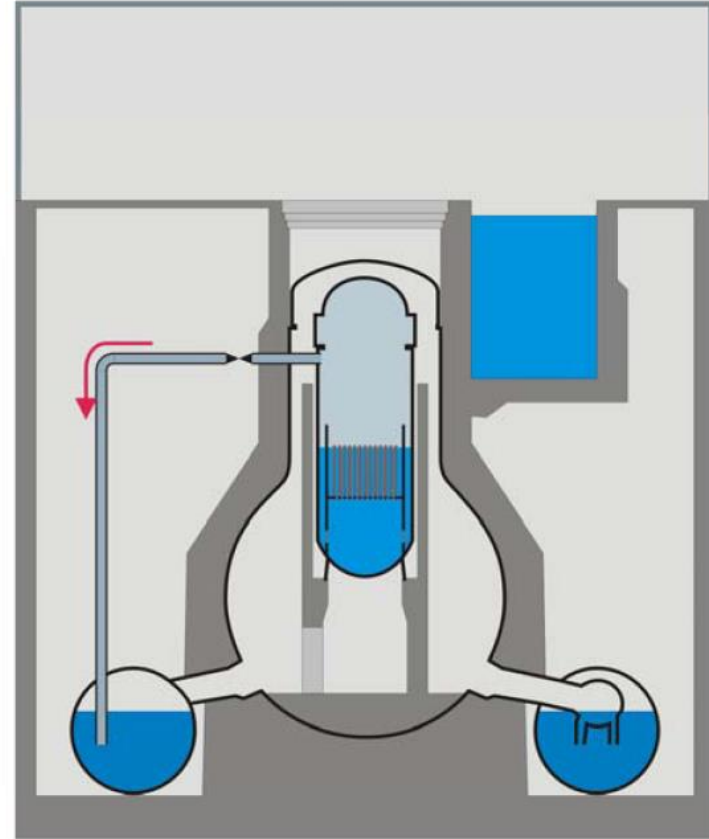


- Flooding of diesel generators.
 - All form of power supply – internal and external - are lost (common cause of failure!).
 - Only batteries are still available.
- ECCS is lost, only the **reactor core isolation pump** mechanically driven by a small steam turbine is available
 - Battery power still needed for turbine auxiliaries.
 - No heat removal from building → water heats up

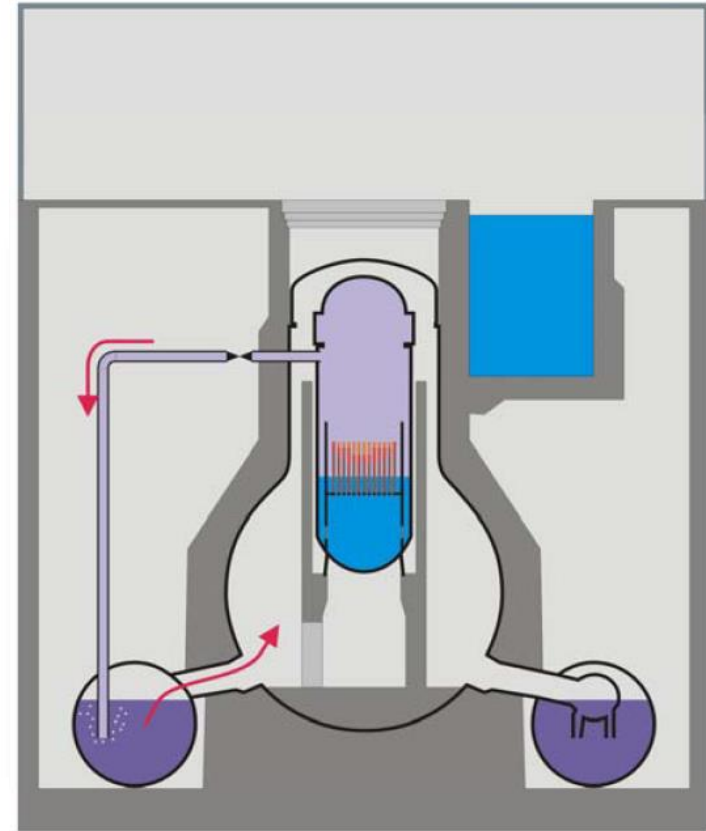


- Unit 1, Match 11 16:36 – Battery empty
- Unit 2, Match 14 13:25 – Pump failure
- Unit 3, Match 13 02:44 – Battery empty

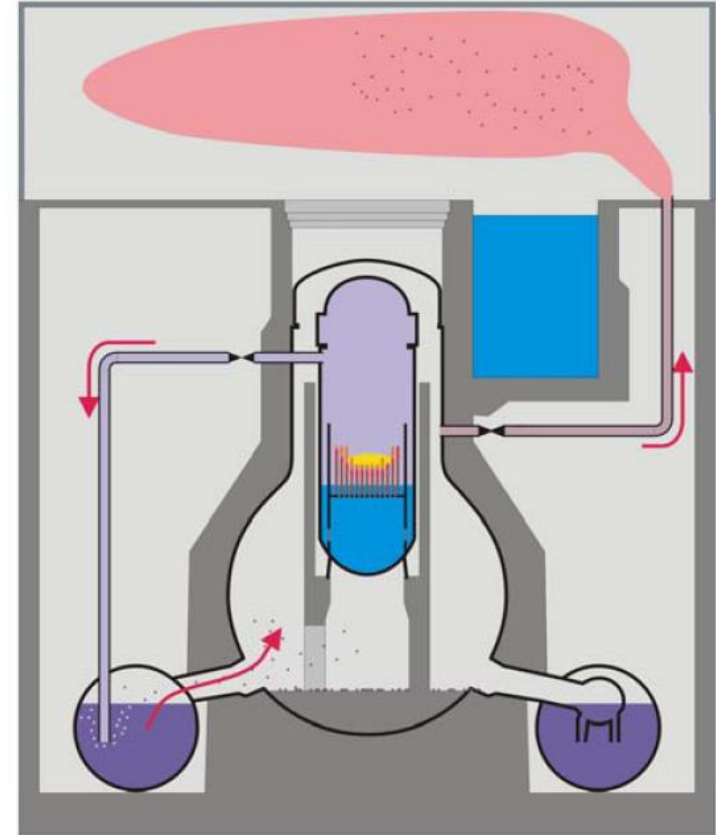
- Decay heat still produces steam, leading to pressure rise:
 - Steam discharged into wet-well (relief valve opening).
 - Liquid level starts to decrease in RPV

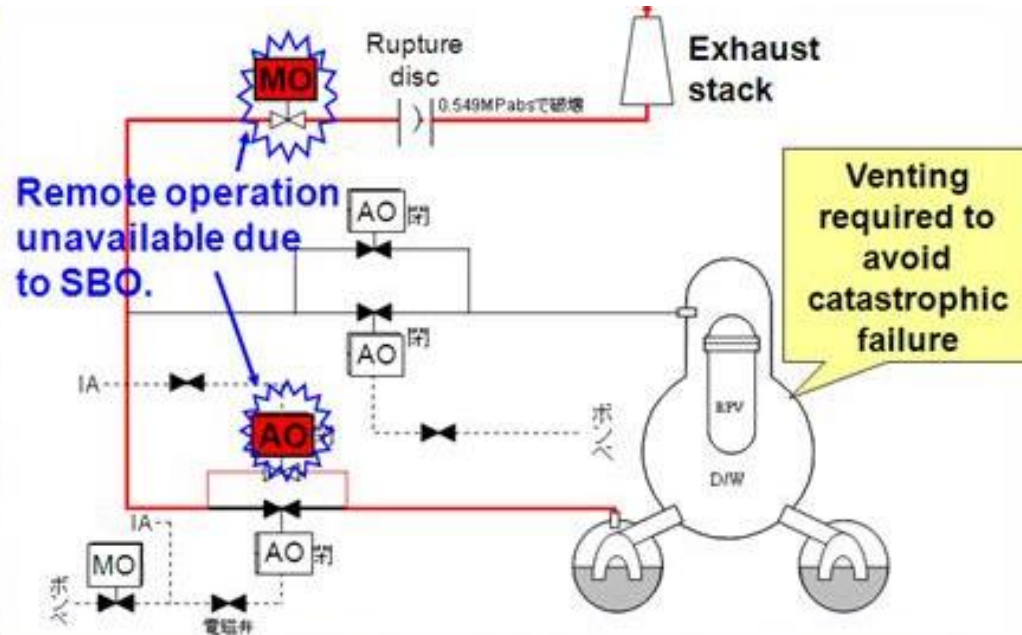


- **Core heat up phase:**
 - Clad temperature rises eventually reaching $\sim 900^{\circ}\text{C}$.
 - Cladding burst and ballooning, release of volatile fission products.
- **Temperature escalation phase**
 - 75% of core cooled only by steam
 - Cladding temperature exceeds 1200°C .
 - Zr oxidation becomes exothermic (300kg to 1000kg of H produced).
 - Produced H is pushed into wetwell.



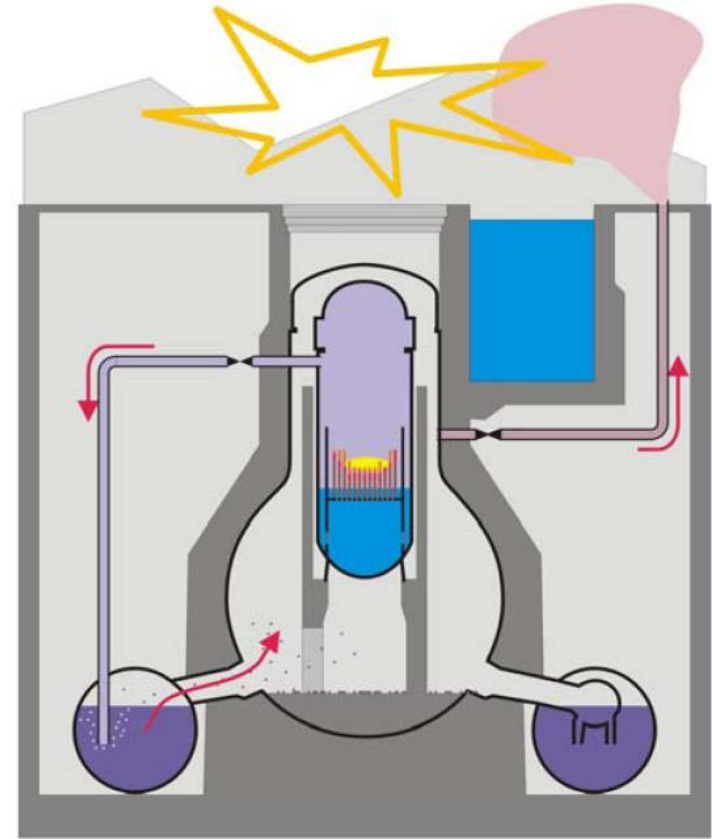
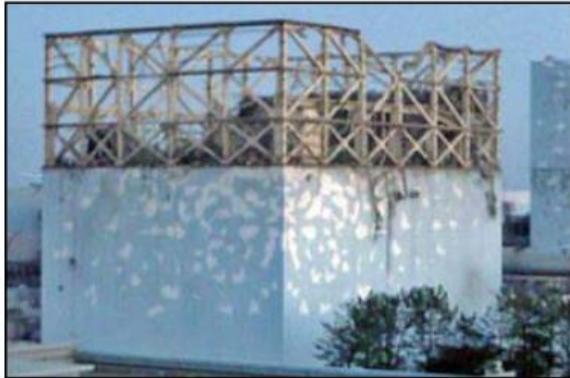
- Significant core melt:
 - Uranium and actinides remain in core.
 - Gaseous and volatile fission products as well as H are released into wetwell and in part enter the drywell.
 - Drywell filled with inert Nitrogen (avoids H explosions)
- Pressure rise in drywell: it arrives at 8 bar against a design pressure of 5 bar.
- Containment depressurization (manually operated)
 - Unit 1, March 12 04:00
 - Unit 2, March 13 00:00
 - Unit 3, March 13 08:41



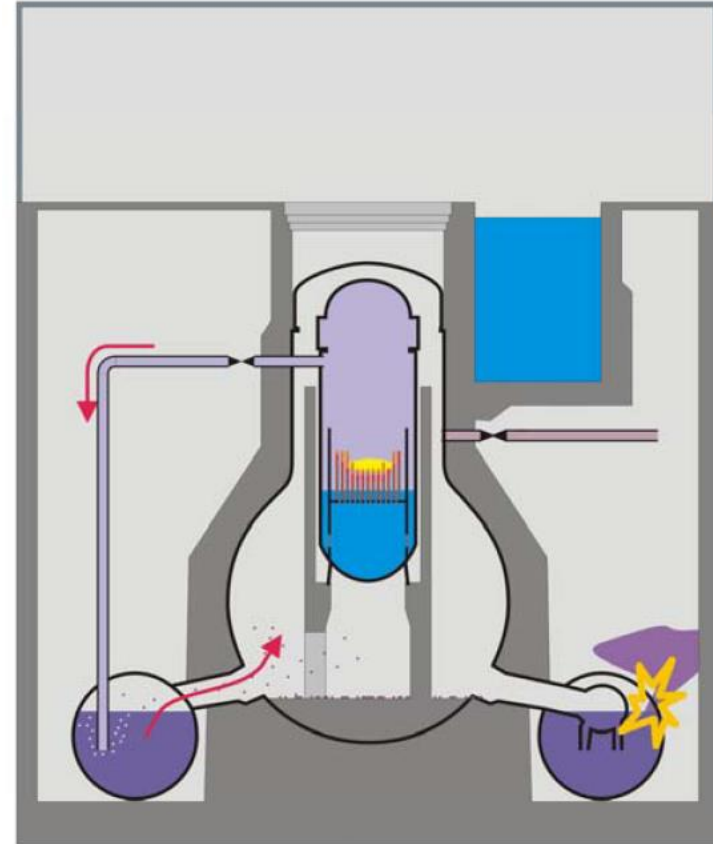


- Six men formed 3 “**last-resort teams**” to manually open 2 valves in **highly-radioactive area**
- Core damage already progressing by this time (3/12 9:04-9:30)

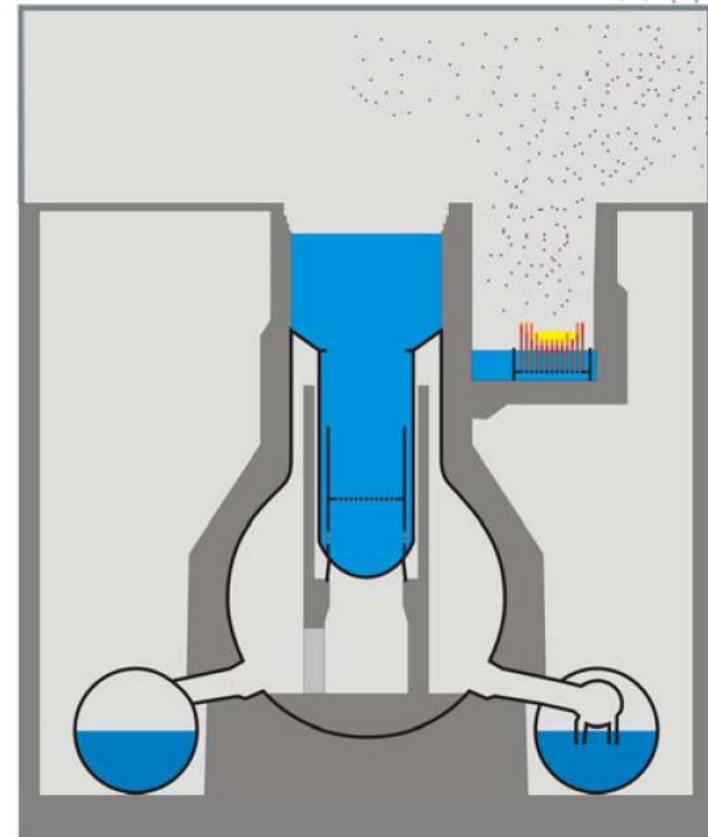
- H explosion inside the reactor service floor.
 - Unit 1 on March 12, 2011, 15:36 JST
 - Unit 3 on March 14, 2011, 11:00 JST
- No H-recombiners were installed.
- Destruction of steel-frame but concrete building remains undamaged.

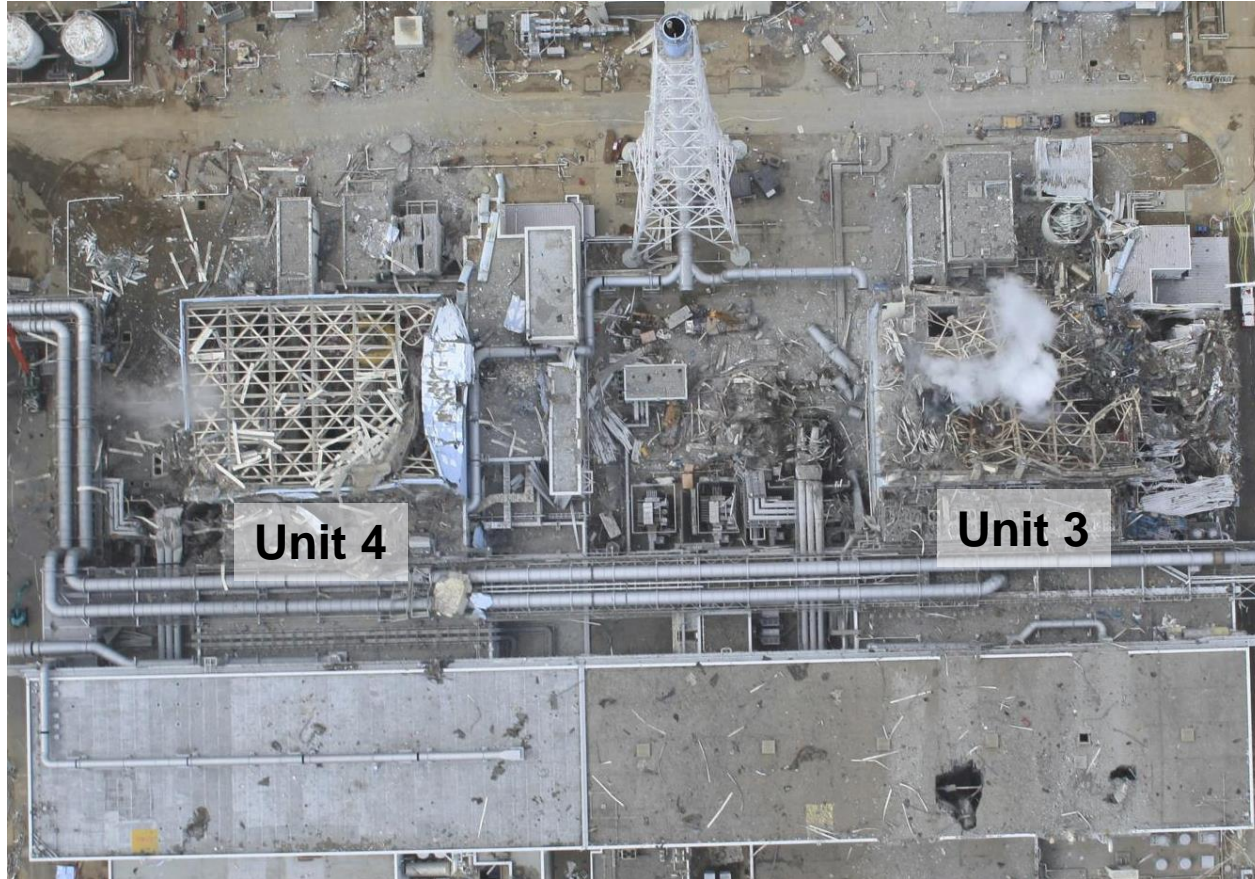


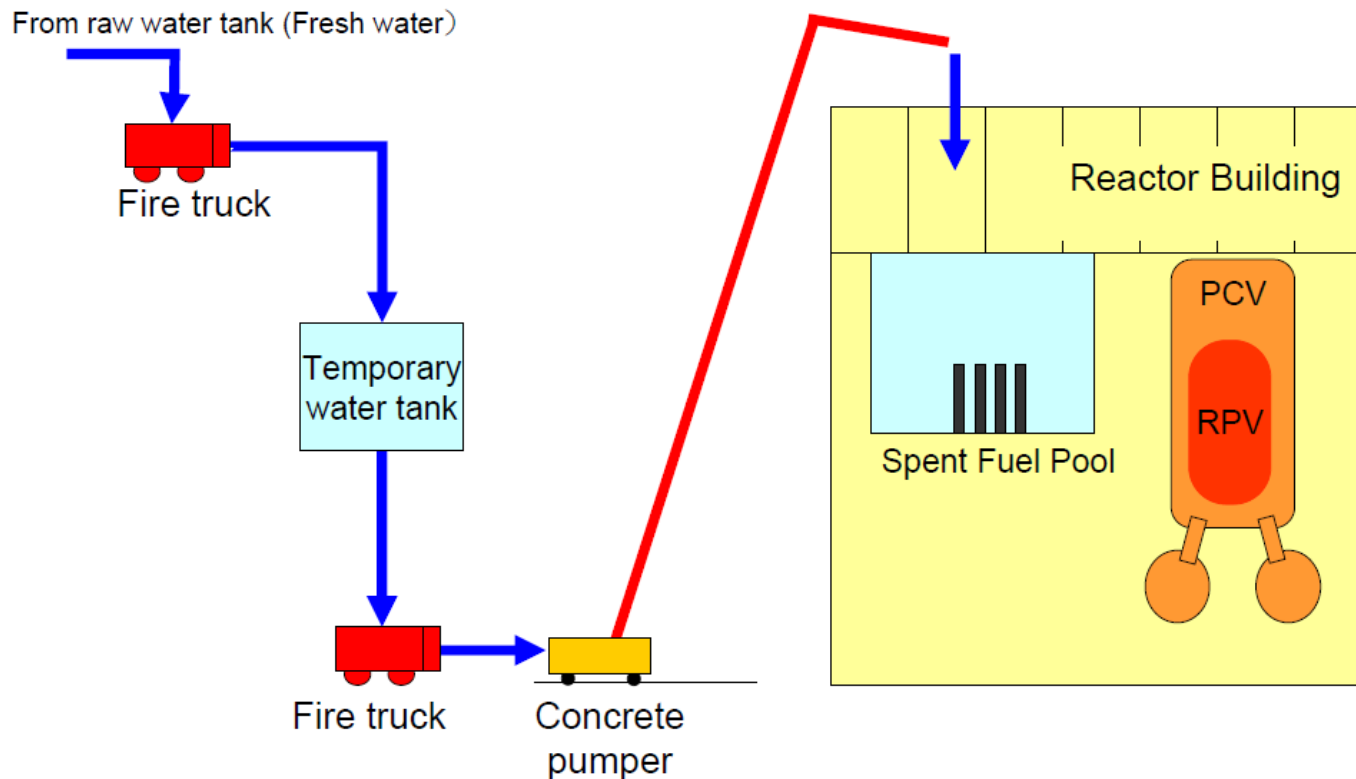
- Explosion with damage of the pressure suppression chamber in Unit 2 on March 15, 2011, 06:10 JST
- Probable damage of drywell due to pressure rise in containment.
- Highly contaminated water (required temporary evacuation).
- Uncontrolled release of gas from containment.



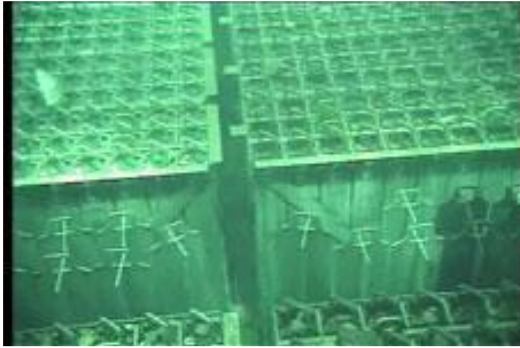
- Explosion with fire in unit 4 close to the fuel assembly storage pool.
 - Not clear why.
 - H leaking from unit 3 through venting stack possible cause.
- Possible danger as entire core was in spent fuel:
 - Expected dryout in 10 days
 - No retention of fission products (no containment).
 - Risk of large releases into environment.











Spent fuel
pond
apparently
showing
no serious
damage

[Nuclear
Emergency
Response
Headquarters,
Government of
Japan]



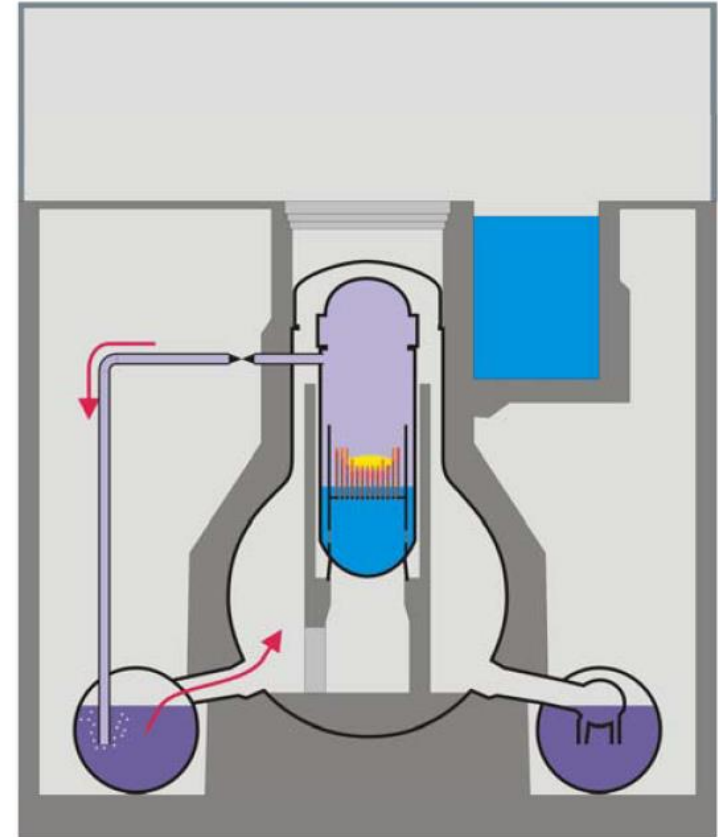
Destruction
of
reactor
building
roof by
hydrogen
explosion

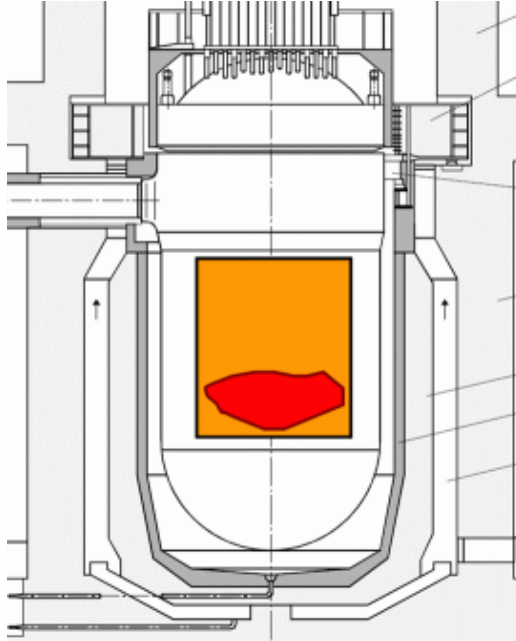
[AREVA]



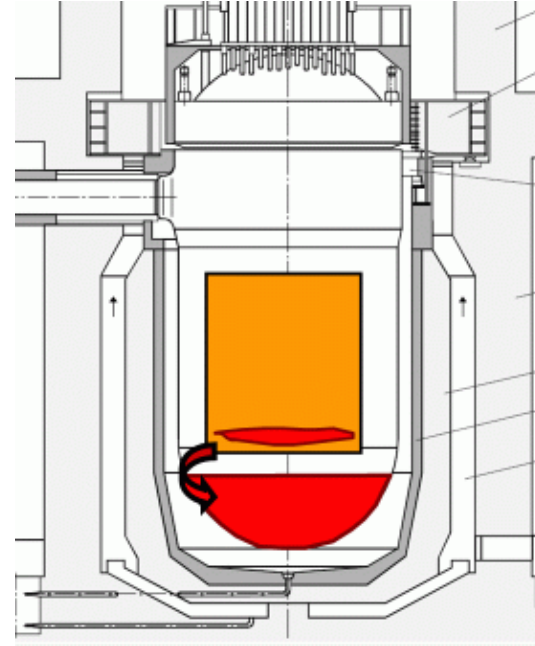
General view of damaged reactor building

- Eventually temperature keeps rising and core melt progresses:
 - At 1800°C cladding melts.
 - At 2500°C core debris bed formation.
 - At 2700°C melting of (U,Zr)O₂ eutectics
- TEPCO decides (maybe too late) to reflood core with seawater to stop melt progression
 - Unit 1, March 12 20:20 → **27h w/o water**
 - Unit 2, March 14 20:33 → **7h w/o water**
 - Unit 3, March 13 09:38 → **7h w/o water**

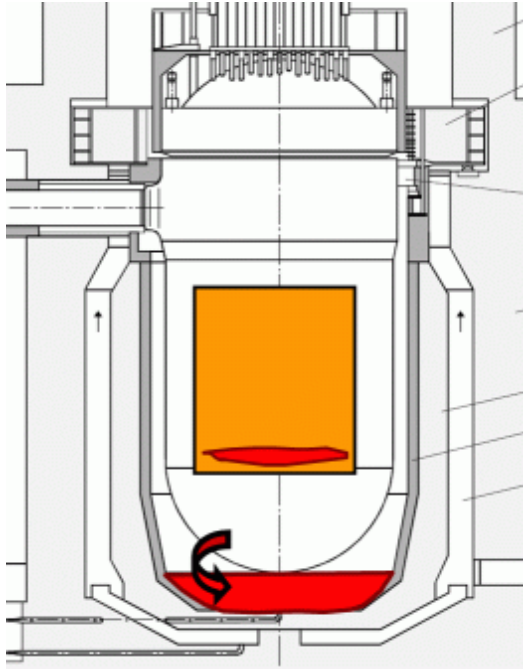




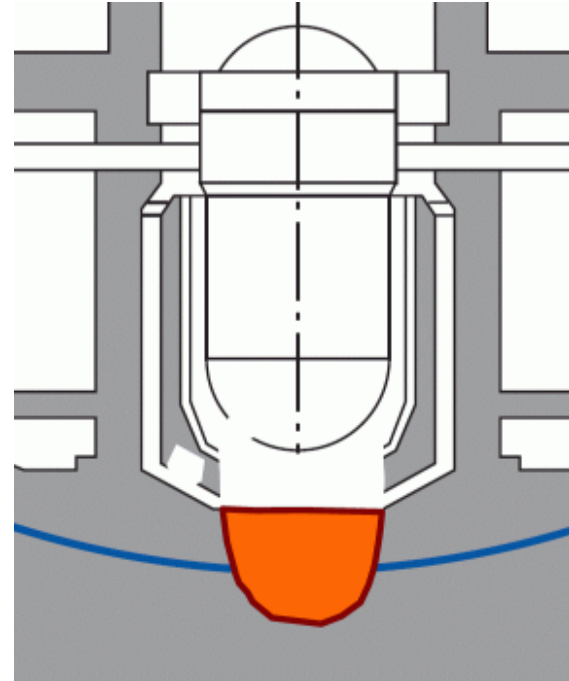
Melting of Reactor Core



Relocation of Molten Core into lower plenum of RPV

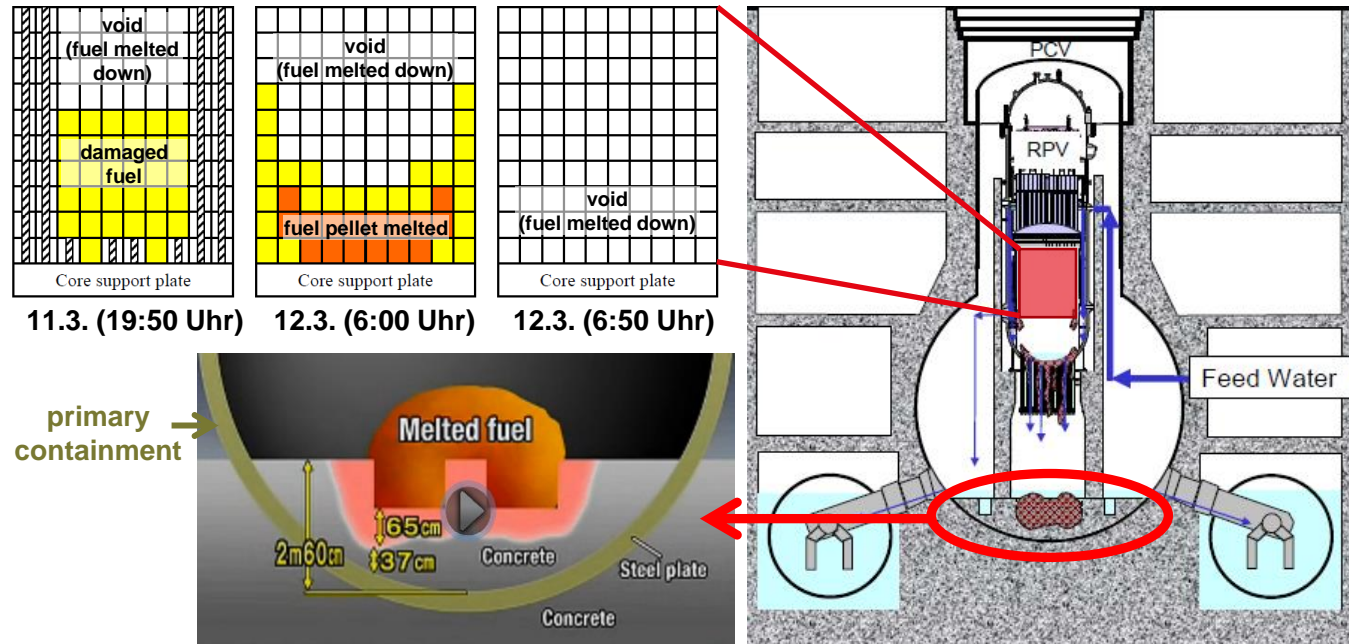


RPV failure and relocation into containment

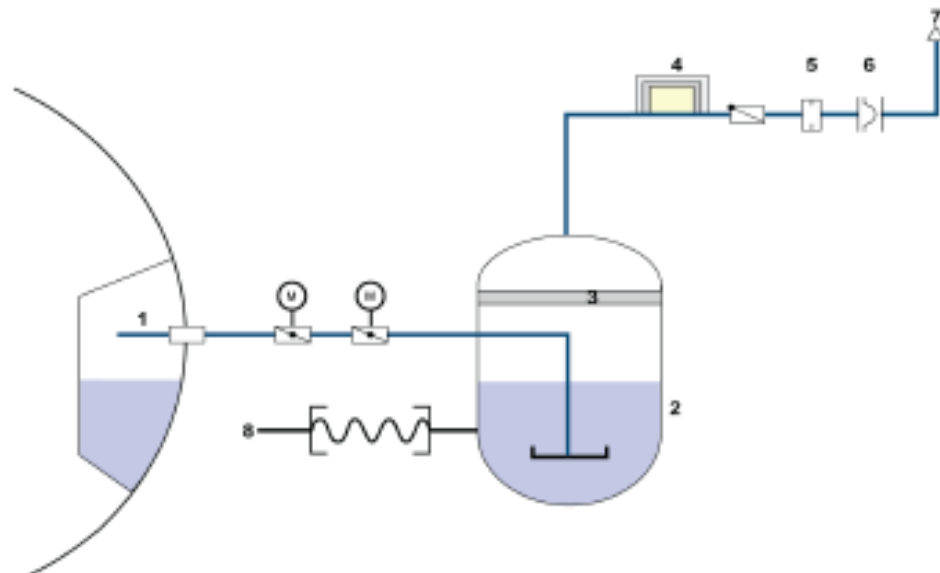


Molten Core breaks through Containment

- Calculations estimate that core did not go all through the bottom.
- However lateral leakage paths are possible so some water might be leaking in the soil.



- Power Plants must be protected against ***simultaneous*** external hazards
- The management of Severe Accidents must take into account **extensive destruction of the surrounding** infrastructure.
- Neighbouring plants can aggravate the situation and physical protection of spent fuel pools must be secured (Unit 4).
- The grace time of a power plant before operator interaction is required should at least be several days (instead of hours)
- Long-term station blackout must be considered, and emergency equipment (mobile pumps, etc.) must be available on-site
- Management of **venting and H-control** is of paramount importance



Containment Pressure Relief System

- | | | |
|-------------------------|-----------------------------|--|
| 1. Condensation Chamber | 4. Dose Measurement Station | 7. Venting Stack |
| 2. Venturi Washer | 5. Orifice | 8. External Feed Water System (Fire Feed Line) |
| 3. Metal Fiber Filter | 6. Blow-Out Disk | |

Picture of the Fort Calhoun Flooding (June 2011)



- Maximal local dose rate of 180 $\mu\text{Sv/h}$ at the outer limit of the 30 km-Zone
- Mostly I-131 and Cs-134/137
- Measures
 - Evacuation (20 km)
 - Sheltering (30 km)
 - Restriction on food for the whole Fukushima prefecture

